

PHYSOR-2004.

The Physics of Fuel Cycles and Advanced Nuclear Systems: Global Developments
April 25-29, 2004, Chicago, IL USA

STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

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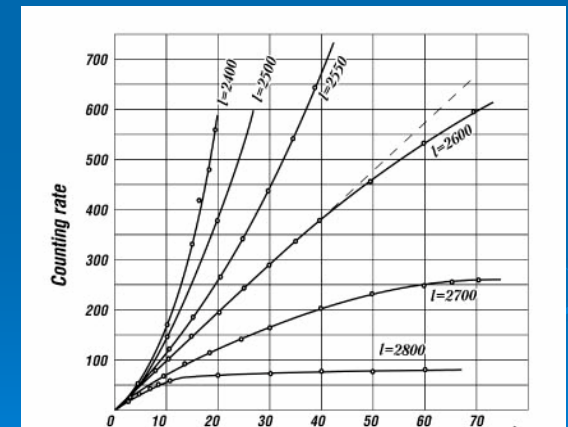
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- 4. Actual Problems**
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STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

FIRST CRITICALITY, 25.12.1946



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- ***Reactor physics is a base of nuclear power, and strength of this base is a necessary condition of nuclear power successful development.***



- ***After the first nuclear criticality had been reached neutron physics experiments played a crucial role in the creation and improvement of nuclear reactors.***

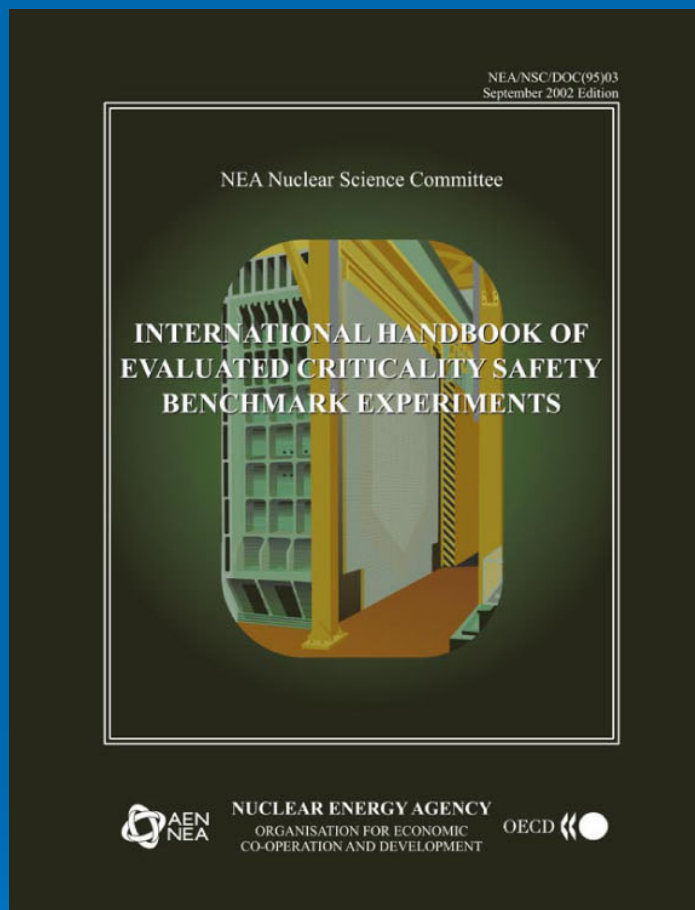


STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

- ***Most experiments were performed (mainly as a necessary stage of reactor design) in the nuclear power “golden age” in 60s and 70s of 20th century, when most of the total of over thousand nuclear reactors have been created worldwide.***



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12 countries,

**About 400 series of experiments
(evaluations),**

**3000 critical and subcritical
configurations,**

**Russian contribution is about 40%
(for systems with ^{235}U of high and
intermediate enrichment – 60%)**

- ***Practically all Russian basic critical experiments for power reactors were performed in RRC Kurchatov Institute and Institute of Physics and Power Engineering (IPPE).***



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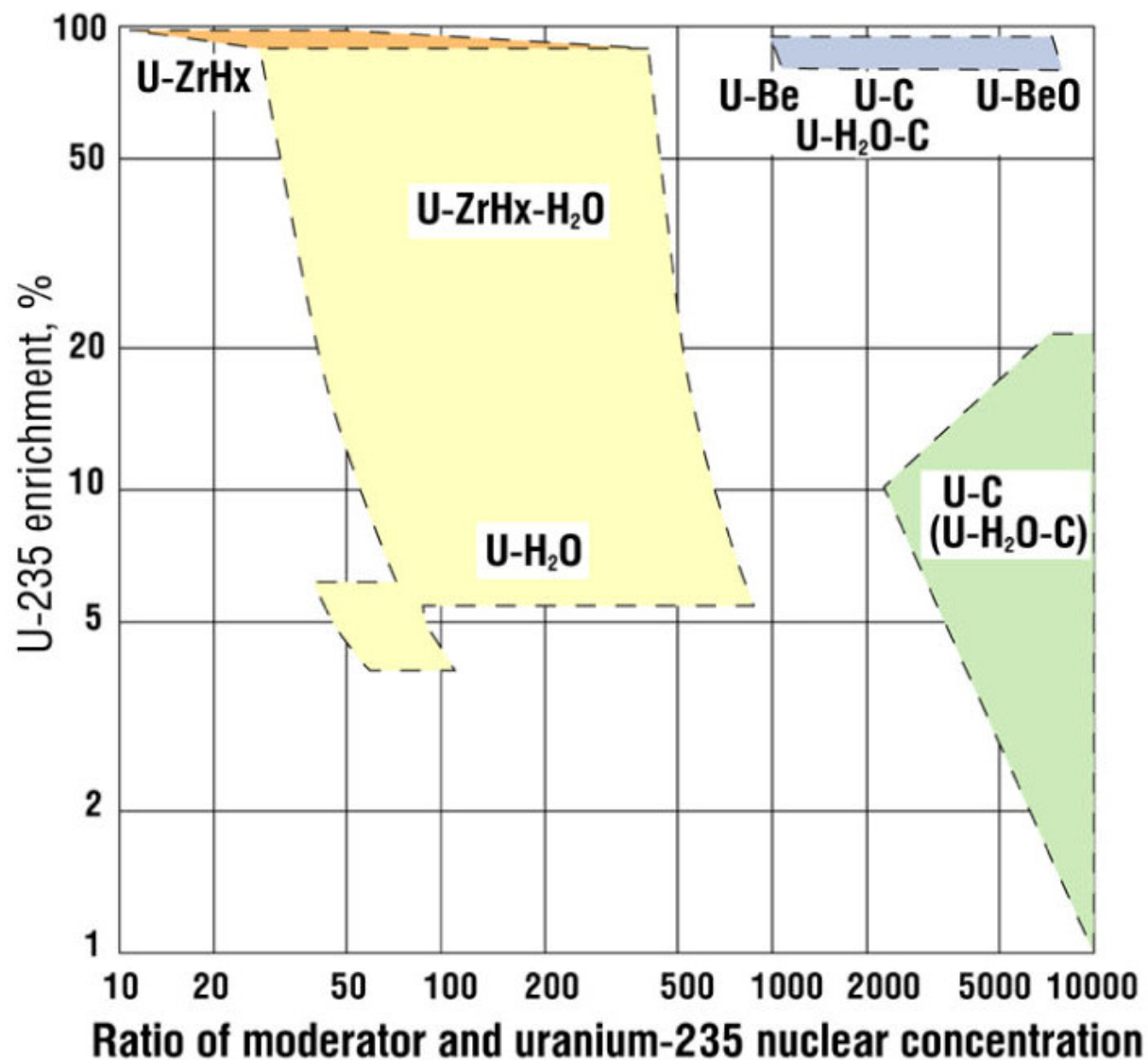
KURCHATOV INSTITUTE – MANY HUNDREDS OF EXPERIMENTS:

- *VVER critical facilities (P, SK-Phys, VVER-1000)*
- *Graphite reactors critical facilities (ASTRA, GROG, RBMK)*
- *Space reactors NARCISS critical facility*
- *Sea reactors critical facilities (SF-1, SF-7, DELTA, KVANT).*



STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

RANGE OF ENRICHMENT AND MODERATOR CHARACTERISTICS FOR CRITICAL EXPERIMENTS IN RRC KURCHATOV INSTITUTE

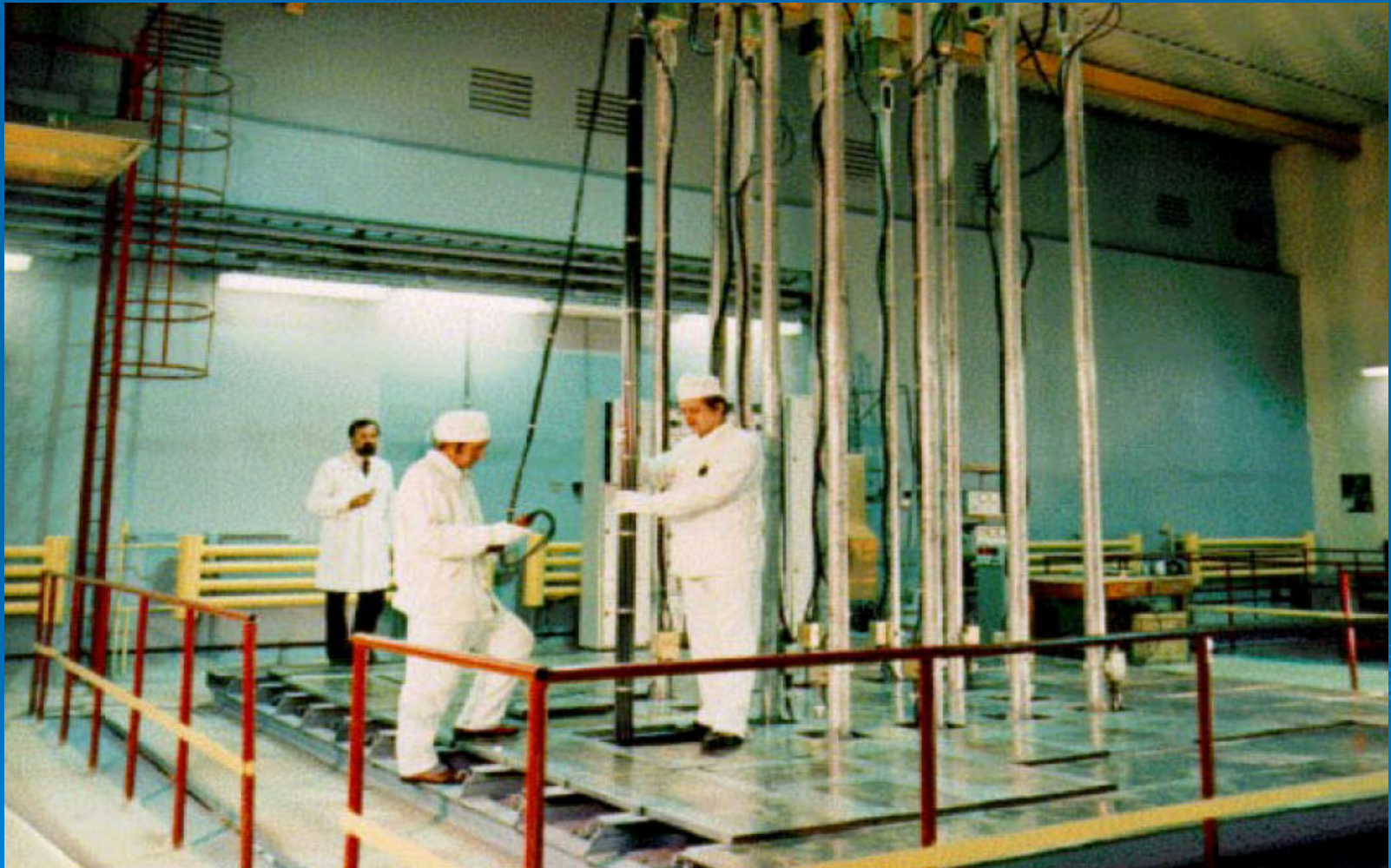


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RBMK CRITICAL ASSEMBLY

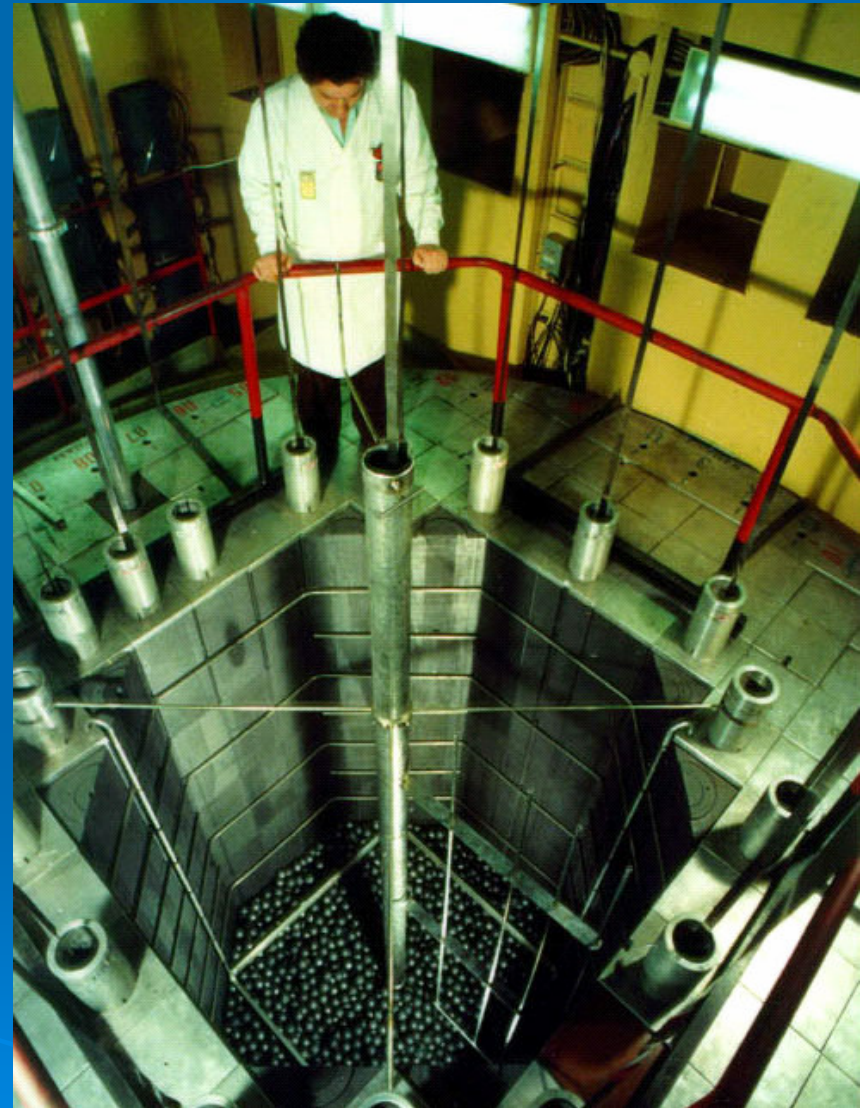


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ASTRA FACILITY



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GROG FACILITY



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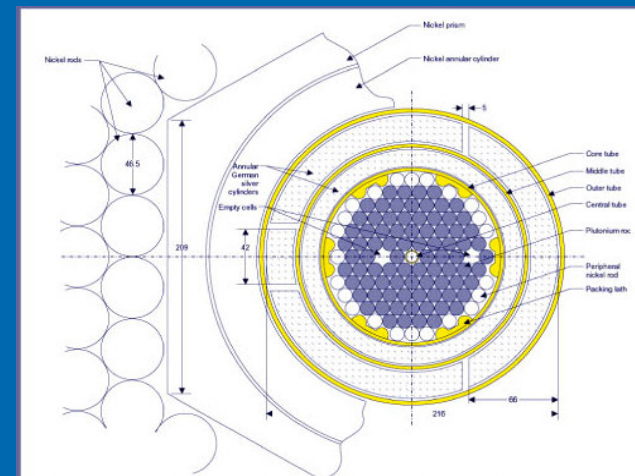
INSTITUTE OF PHYSICS AND POWER ENGINEERING (IPPE):

- *The BFS-1 and BFS-2 critical facilities – an unique experimental facilities for investigations of fast reactor physics, including burning actinides and weapon-grade plutonium utilization (BFS-2 - the world's largest fast critical facility)*
- *MATR-2 critical facility – high-temperature high-pressure uranium-water facility.*



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BFS-1 CRITICAL FACILITY IN IPPE



Critical Assembly with Plutonium rods

Experiments for:

- fast neutron systems
- uranyl nitrate solutions (reprocessing facilities)
- space and sea reactors

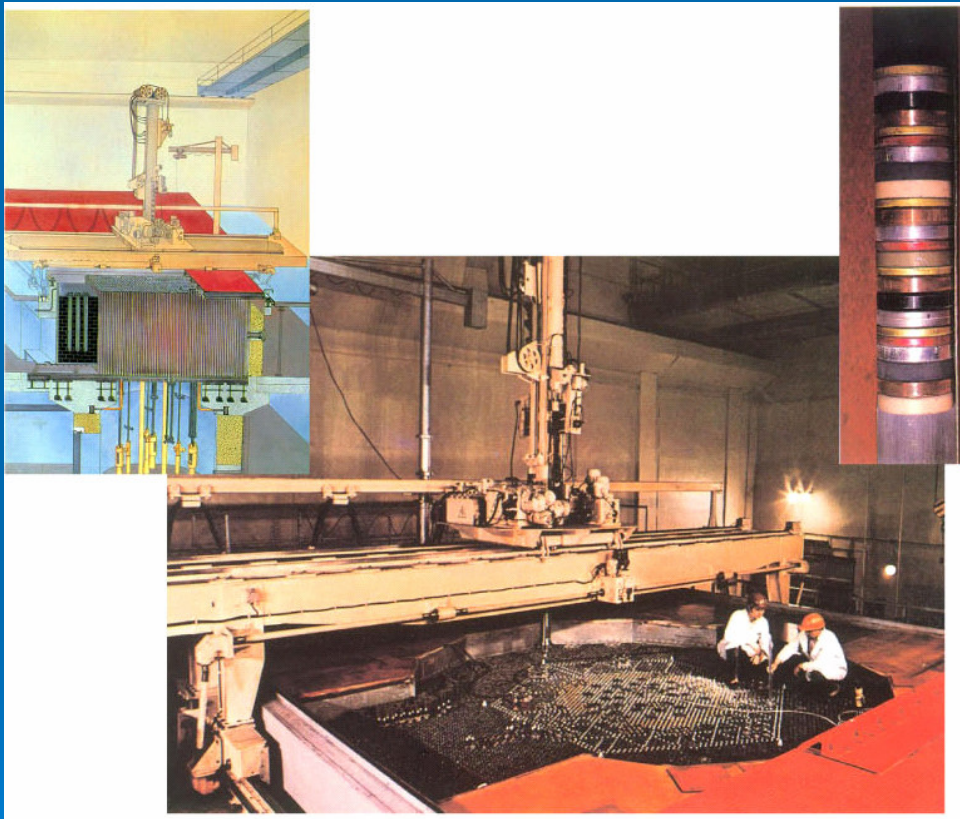


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BFS-2 CRITICAL FACILITY IN IPPE



Directions of works:

- BN-600 reactor core with MOX-fuel
- BN-800
- lead coolant reactors
- actinides transmutation
- neutron data for lead and bismuth



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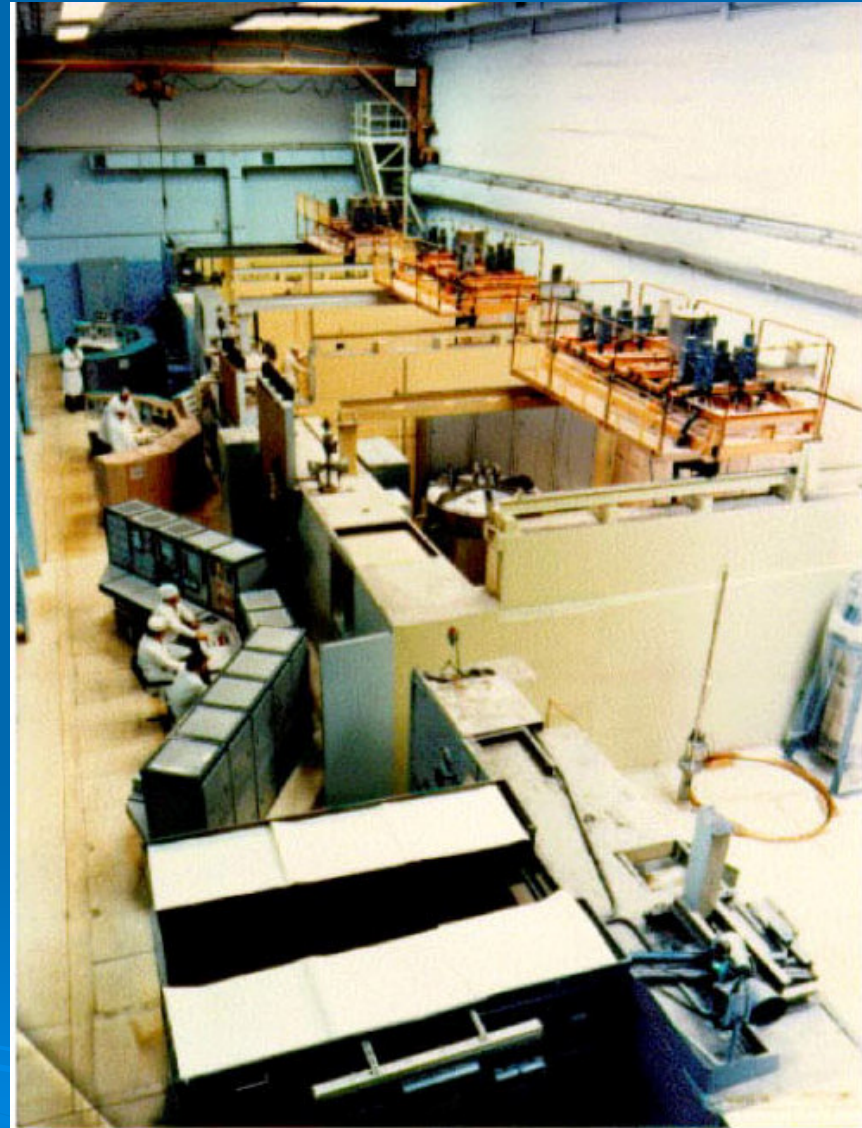
STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

- *Kurchatov Institute and IPPE still preserve capabilities for conducting the reactor experiments: qualified specialists, nuclear fuel of a wide range of compositions, forms and enrichment, facilities, equipment, etc.*
- *Series of experimental programs were performed in the last years, including those ordered by foreign laboratories.*



STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

Russian Nuclear Centres Still Preserve the Capabilities for New Critical Experiments



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➤ ***Today's situation:***

Only restricted experimental modeling of reactors is possible due to financial problems



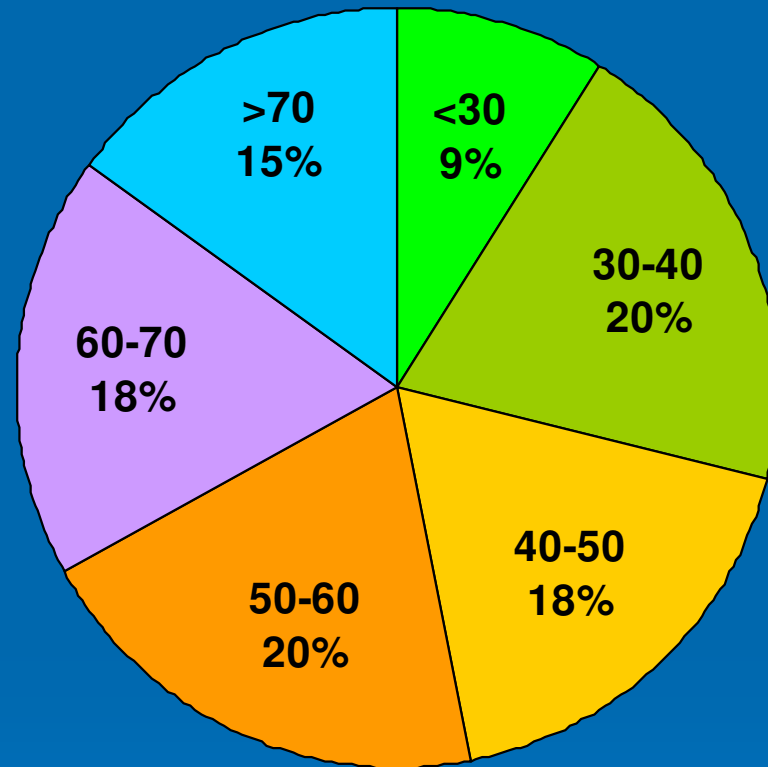
➤ ***Today's situation :***

The average age of reactor scientists increases, especially in experimental labs.



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**STAFF AGE
DISTRIBUTION IN THE
REACTOR PHYSICS
DEPARTMENT.
AVERAGE AGE - 52**



- ***Today's situation:***
Rapid evolution of hardware and system software and the following development of mathematical methods and applied codes.



- ***Despite of any progress in calculations the basic experiments will be always necessary for reliable basing of main safety and economics parameters of nuclear reactors.***



STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

➤ *Actual applied problems:*

- *Support of safety and lifetime extension of operated NPP*
- *Improvement of fuel cycles and economy of nuclear power*
- *Investigation and development of innovative concepts*

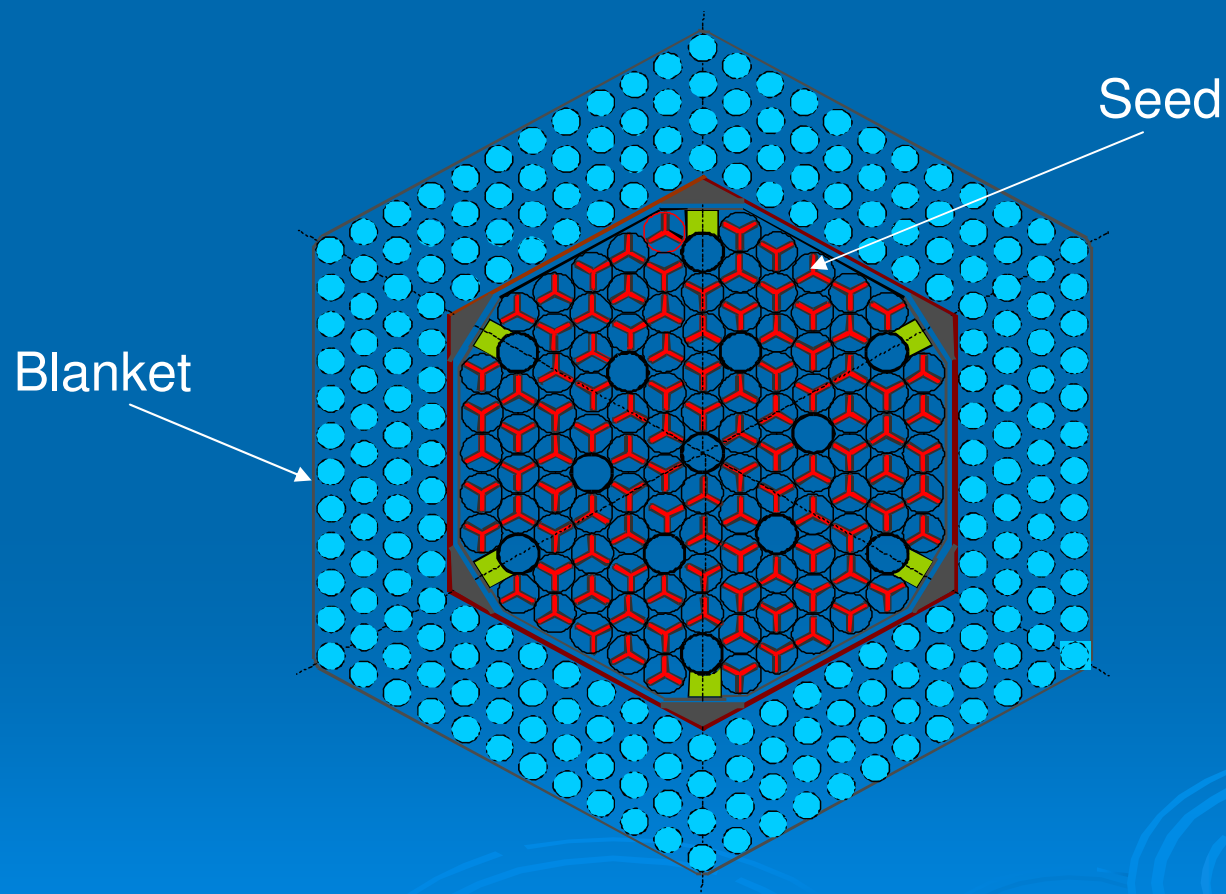


STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

- ***Investigation and development of innovative concepts:***
 - ***power reactors of next generation,***
 - ***power reactors with MOX fuel,***
 - ***non-proliferative reactors with thorium fuel cycle,***
 - ***reactors with heavy metal coolant and breeding ratio closed to unity,***
 - ***reactors of small and intermediate power for local power consumers and outermost regions,***
 - ***reactors and accelerator driven subcritical systems for fuel cycle closing,***
 - ***etc***

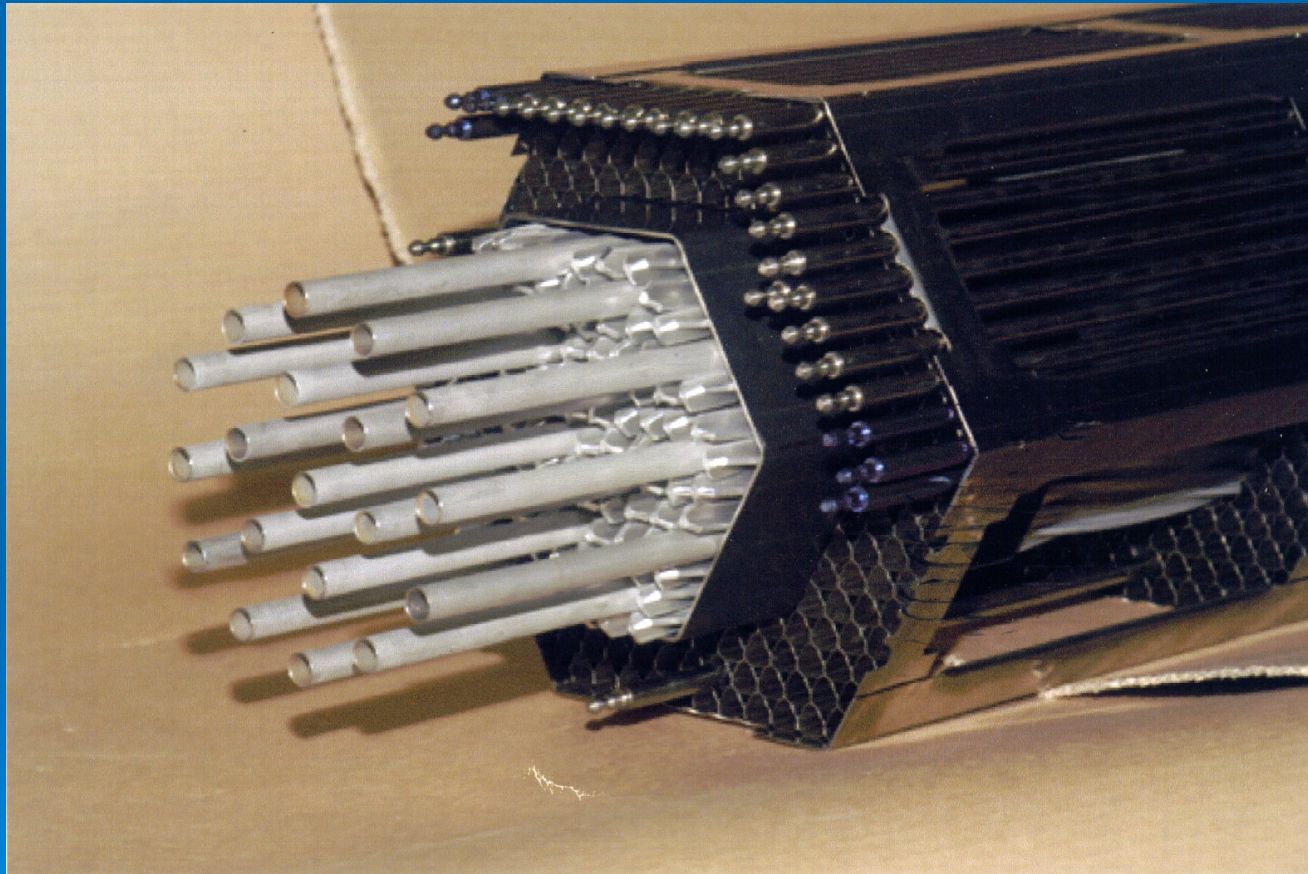


DESIGN OF VVER-T FUEL ASSEMBLY



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MOCK-UP OF VVER-T SEED/BLANKET FUEL ASSEMBLY

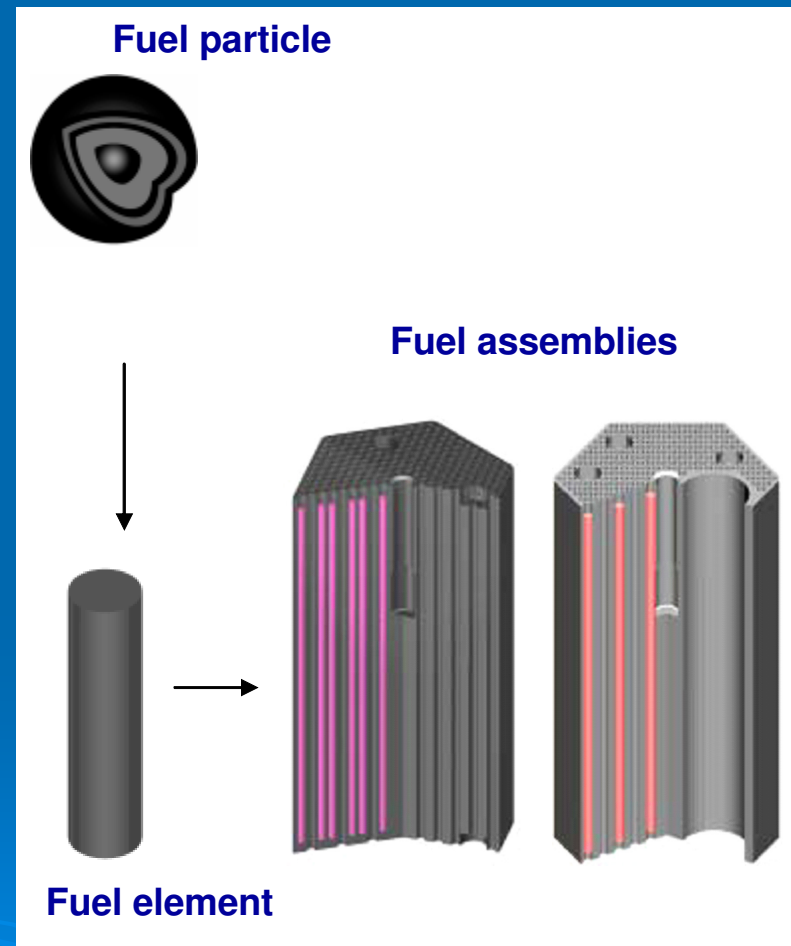
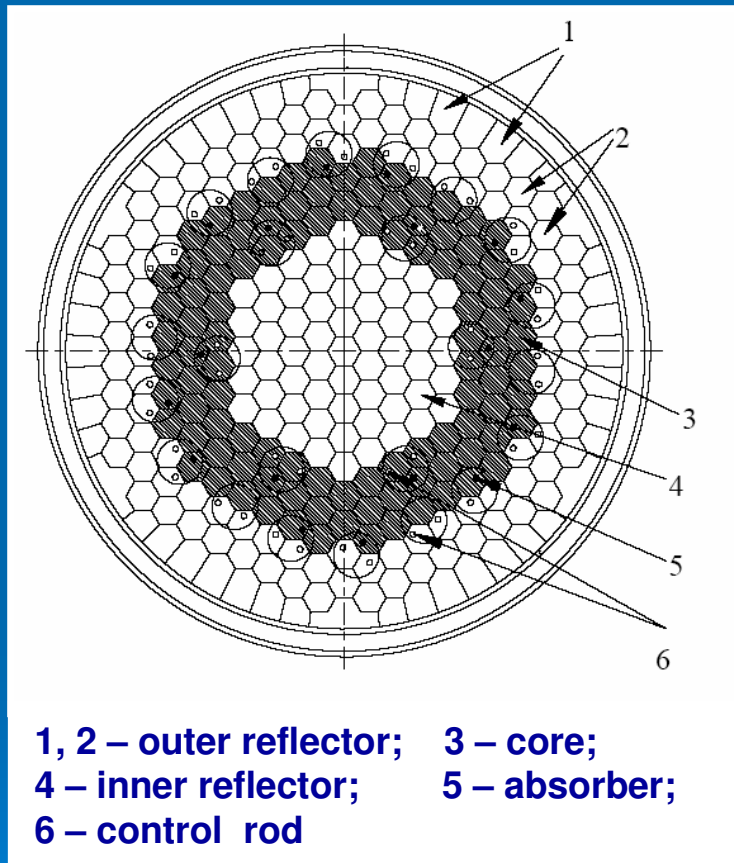


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GAS TURBINE MODULAR HELIUM REACTOR (GT-MHR)



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- ***The development of innovative concepts must be based on the modern approaches, which have to be verified by the evaluated “old” benchmark experiments and new ones.***



- ***“Program of strengthening physical base of nuclear power”:***
Foundation for reliable and precise prediction of neutronic parameters of reactor innovative concepts and for improvement of existent reactors and lifetime extension.



- ***It is strongly necessary to collect, evaluate, and conserve all past experimental experience and data obtained.***

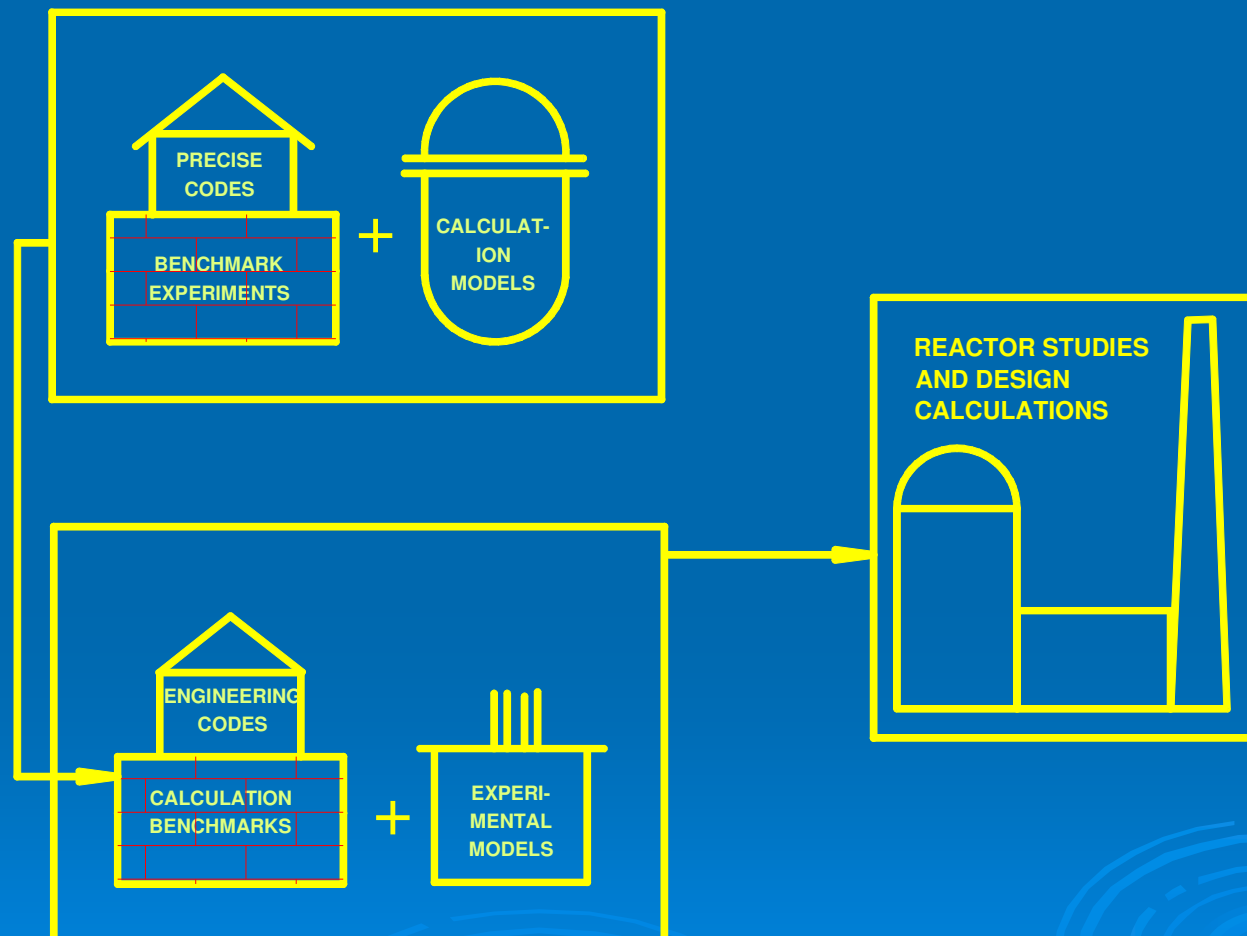


- ***Calculation benchmarks - representative test models of innovative reactors and results of calculations with precise codes, which are validated by all appropriate certified experiments.***



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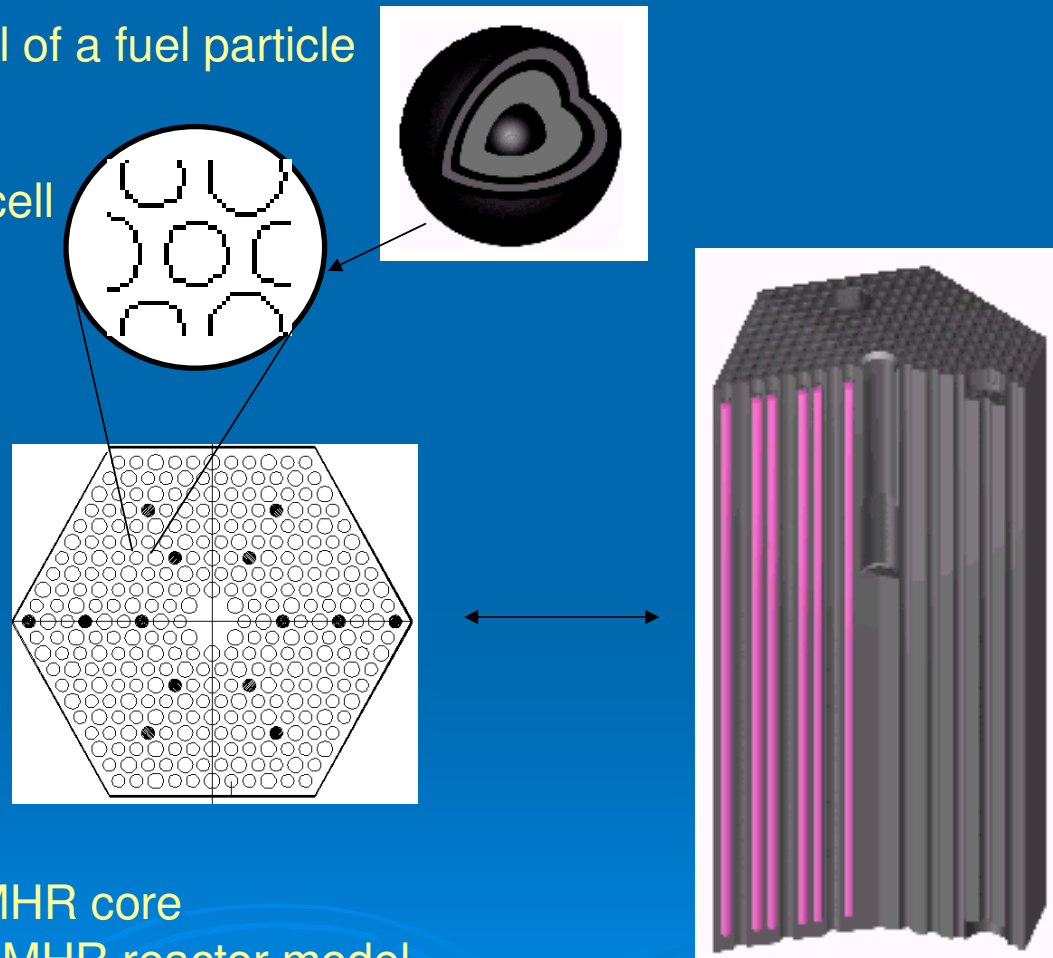
PHYLOSOPHY OF BENCHMARKS USING



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GT-MHR Computational Benchmarks

- Elementary cell of a fuel particle
- Fuel compact cell
- Fuel block



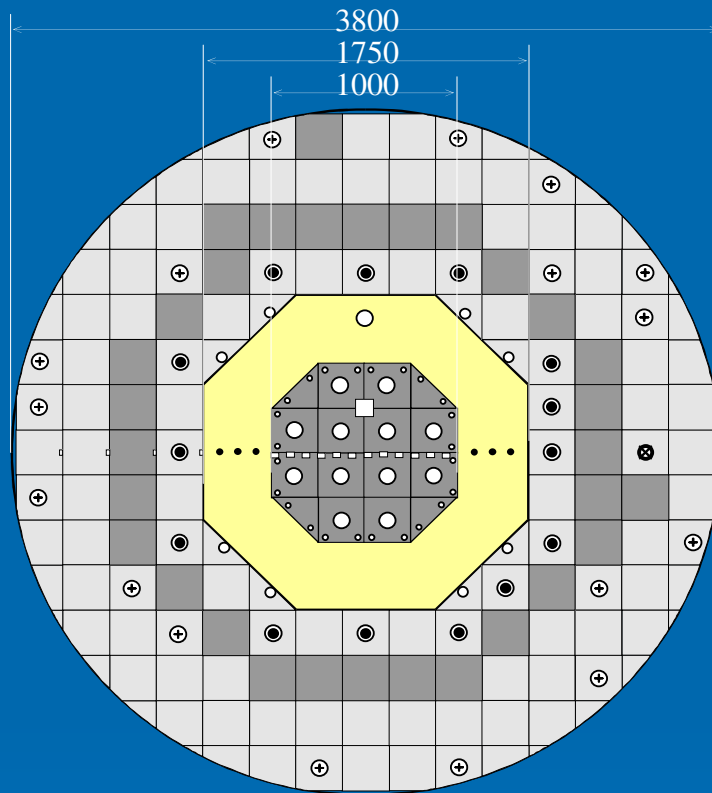
- Layer of GT-MHR core
- Full-scale GT-MHR reactor model



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BENCHMARK EXPERIMENT FOR GT-MHR IN ASTRA FACILITY WITH ANNUAL PEBBLE BED CORE

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Code MCU – Monte Carlo universal

Code UNK (cell)

Collision probabilities method

1D, 2D

Energy groups:

24 – fast

7000 – resolved resonances region

65 – thermal

Code UNK GRO (reactor)

Method of characteristics

3D (arbitrary geometry)



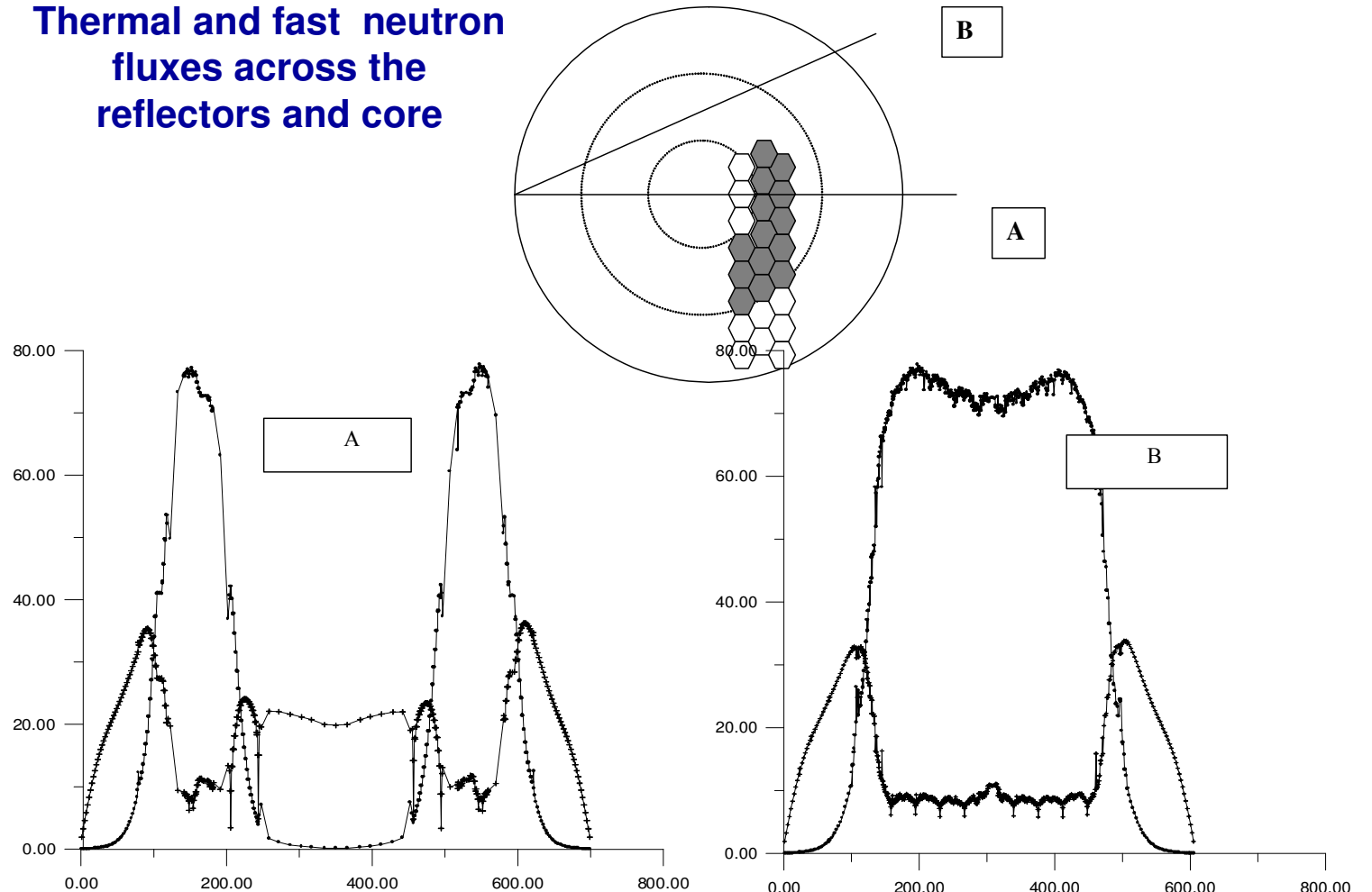
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FULL SCALE PIN-BY-PIN TRANSPORT CALCULATIONS OF GT-MHR (UNK CODE)

Thermal and fast neutron
fluxes across the
reflectors and core

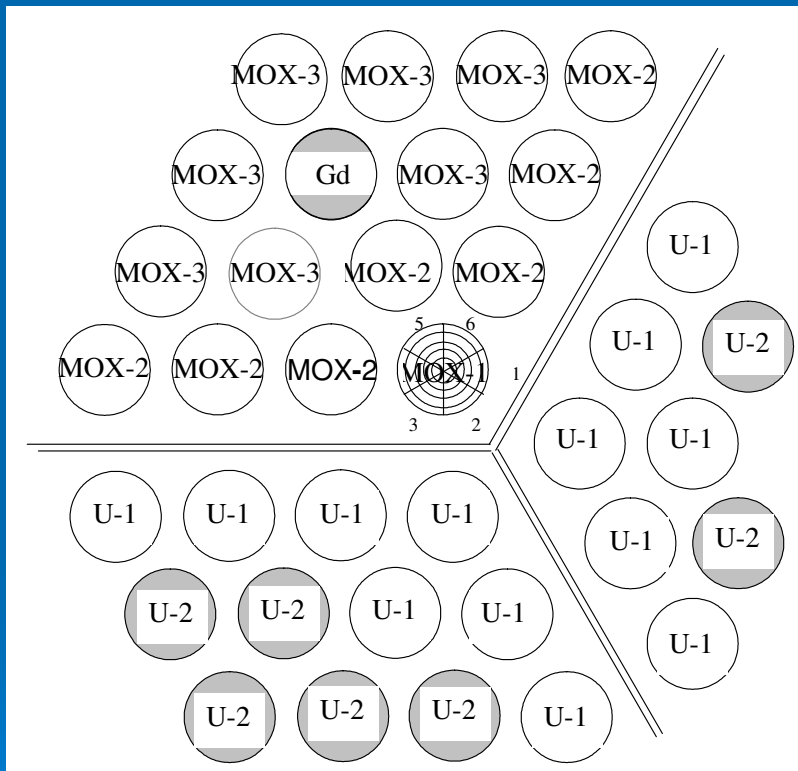


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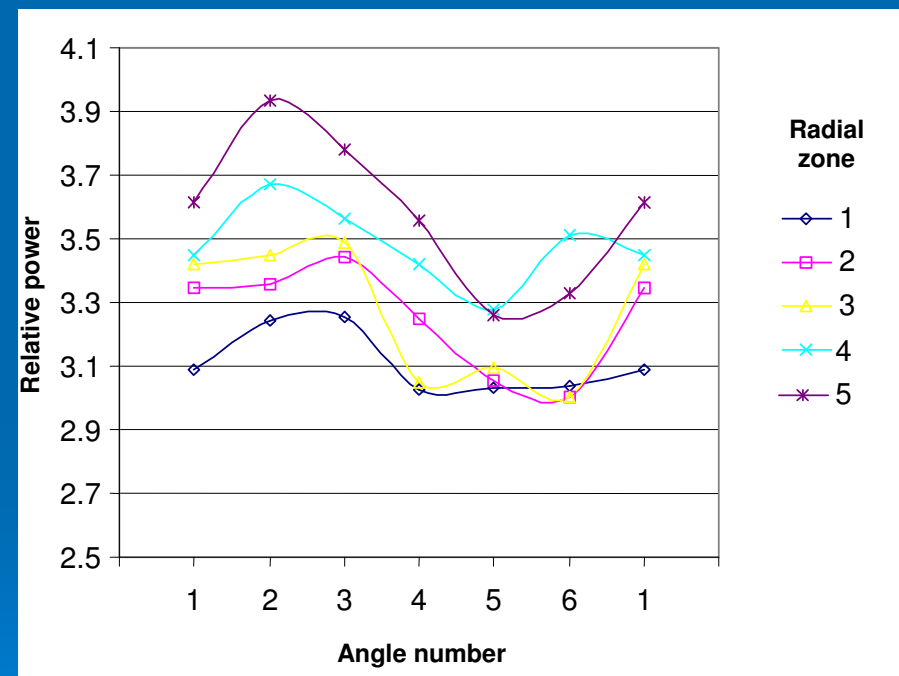
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RADIAL-AZIMUTH POWER DISTRIBUTION IN THE CORNER FUEL ELEMENT (VVER-1000)

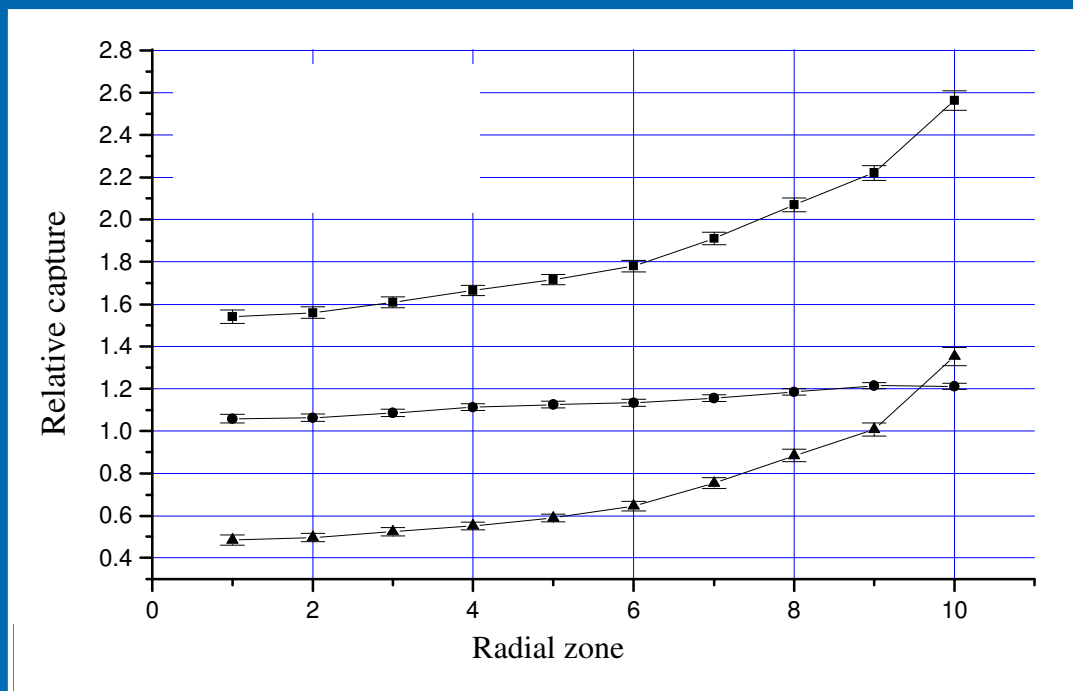
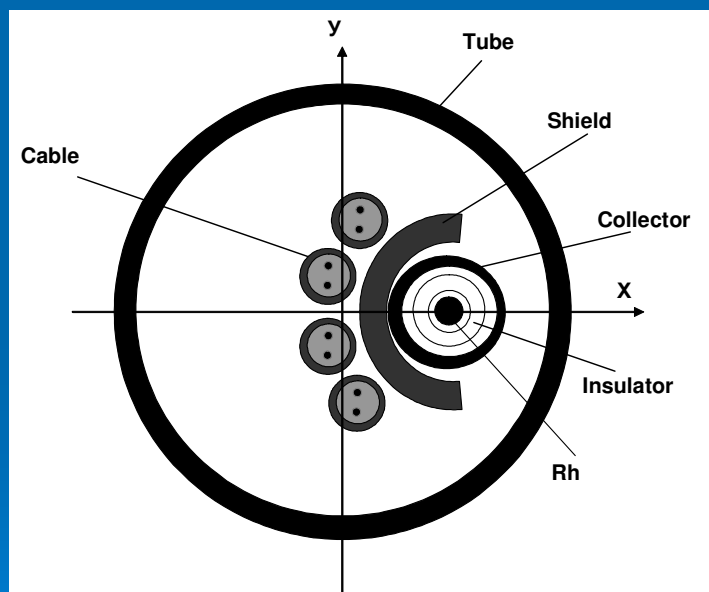


MCU calculation



MCU CALCULATION OF Rh SELF-POWERED DETECTOR

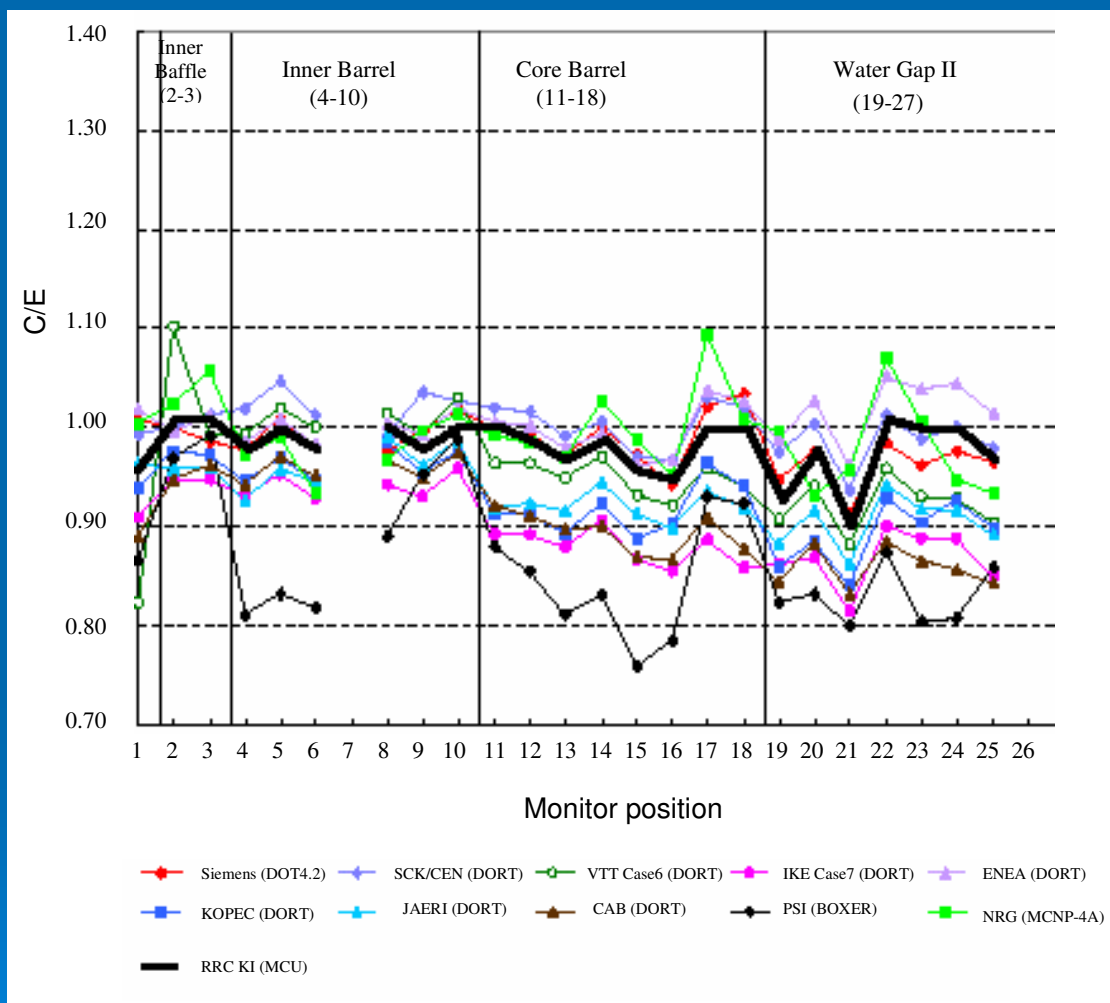
Calculation model of self-powered detector



Radial distribution of neutron capture in Rh wire

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VENUS INTERCOMPARISON. RATIO OF CALCULATED (C) AND EXPERIMENTAL (E) VALUES OF EFFECTIVE FISSION NEUTRON FLUX ($^{238}\text{U}(n,f)$)



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PARALLEL COMPUTATION

➤ CODE LUCKY

***NEUTRON AND GAMMA TRANSPORT
3D XYZ GEOMETRY
DISCRETE ORDINATES***

➤ COMPUTER MBC-1000M

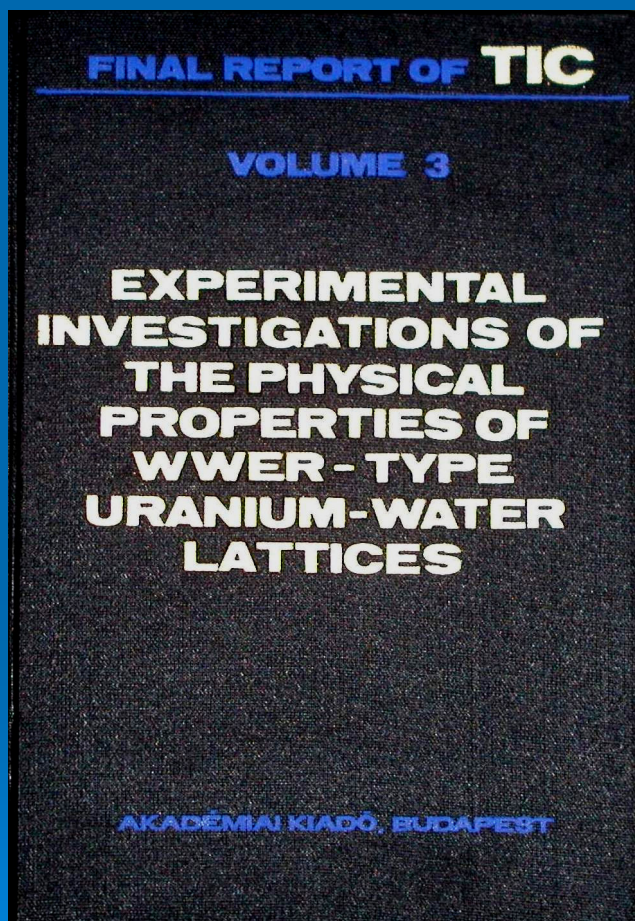
***700 PROCESSORS (667 MHz)
 P_3S_8 APPROXIMATION
40 ENERGY GROUPS
 $2 \cdot 10^8$ MESH INTERVALS***

<600 MIN



STRENGTHENING OF PHYSICAL BASE OF NUCLEAR POWER

TEMPORARY INTERNATIONAL COLLECTIVE (TIC) FOR JOINT RESEARCH INTO THE PHYSICS OF WWER-TYPE REACTORS



10 countries

300 experiments

Theoretical studies

Calculation methods

Codes

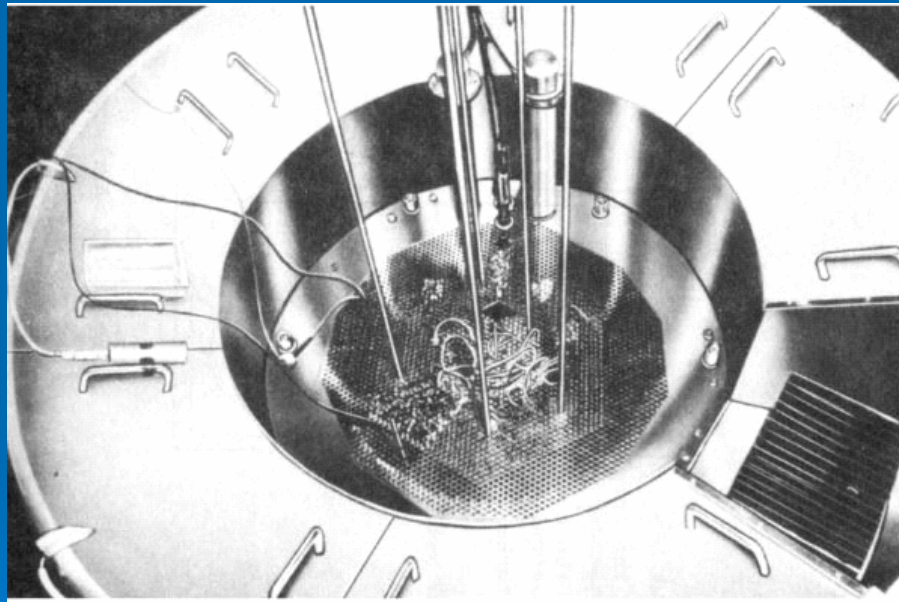


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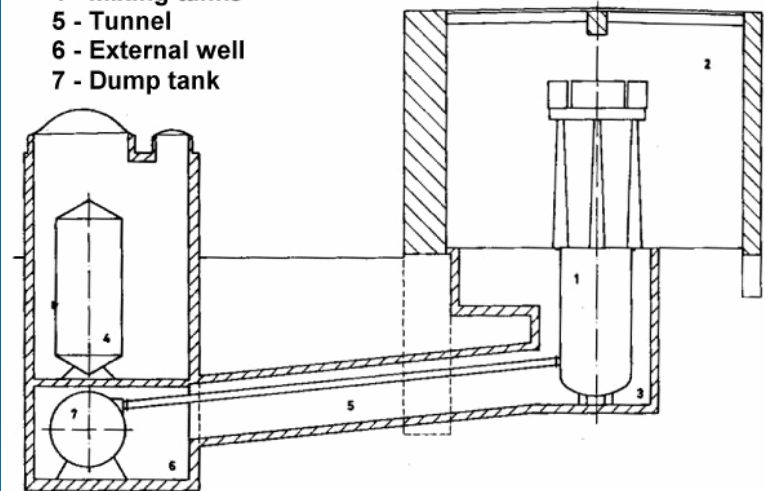
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CRITICAL ASSEMBLY ZR-6 (HUNGARY)



- 1 - Core tank
- 2 - Hall of the critical assembly
- 3 - Well of the critical assembly
- 4 - Mixing tanks
- 5 - Tunnel
- 6 - External well
- 7 - Dump tank

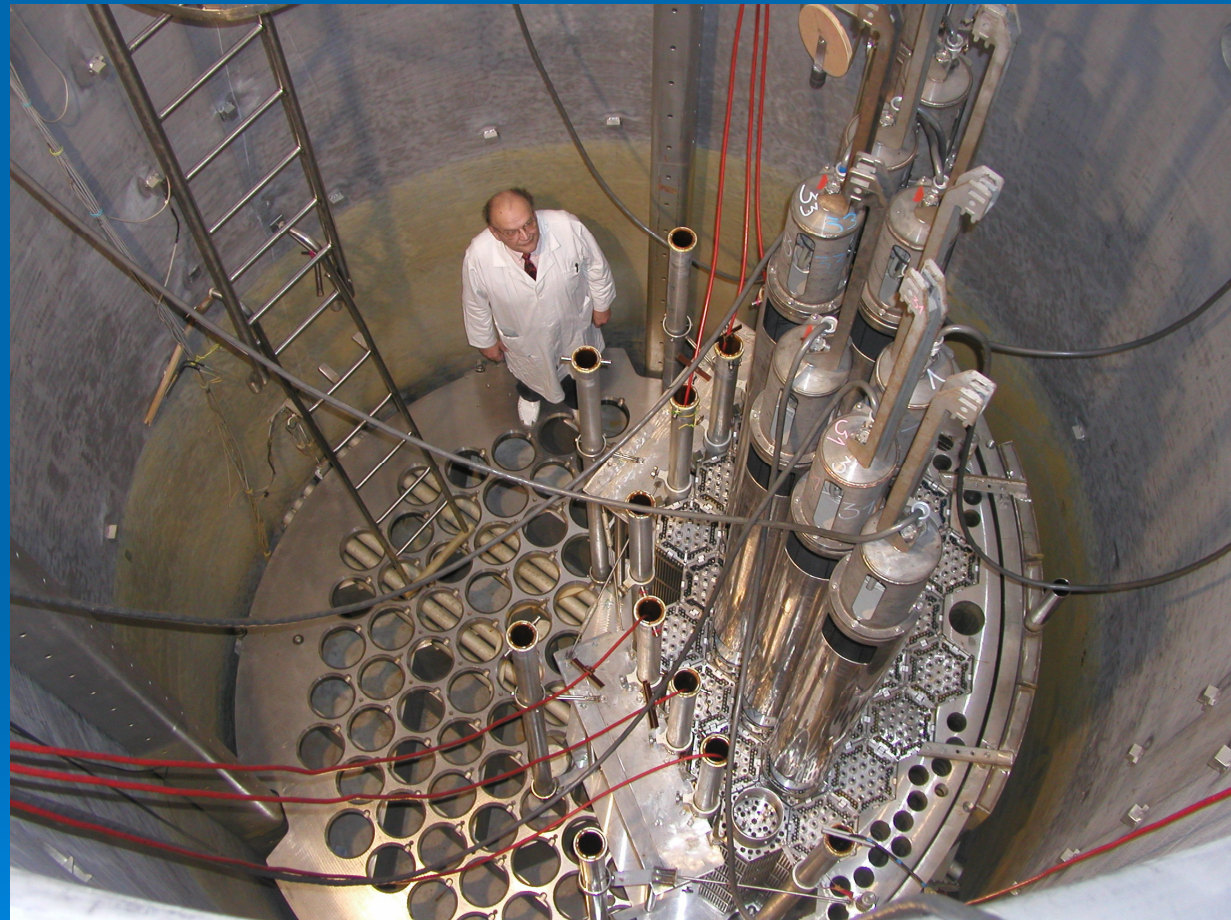


Arrangement of the equipment of the critical assembly

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CRITICAL ASSEMBLY LR-0 (Czech Republic)

Full-scale
Mock-up of
VVER-1000

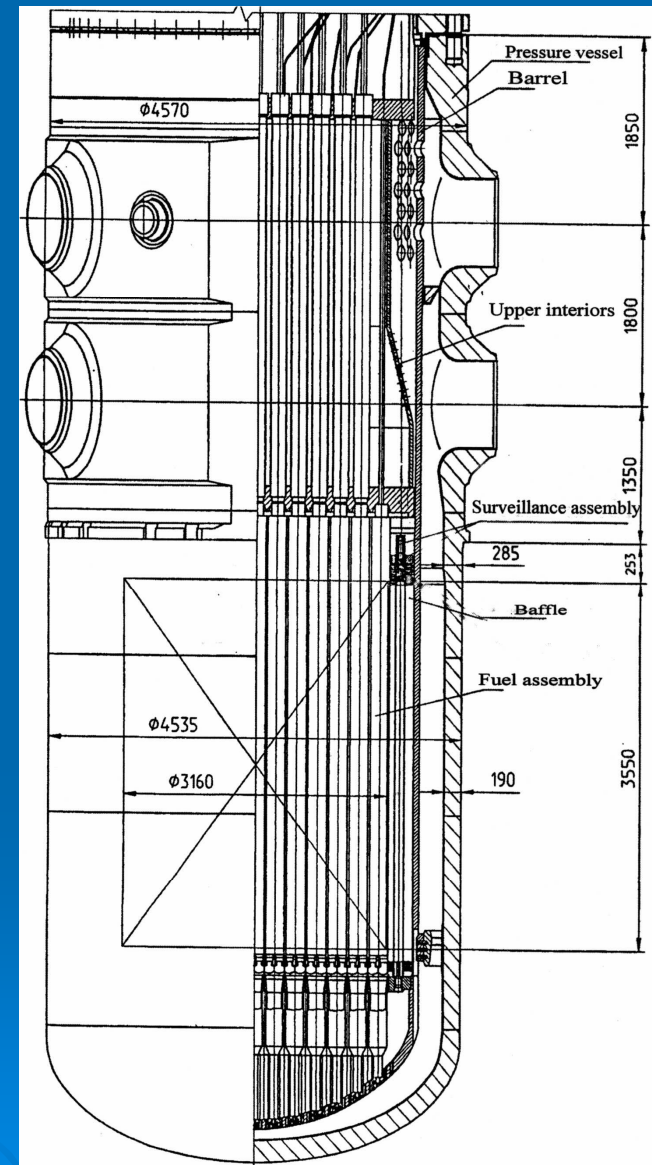


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REACTOR VVER-1000

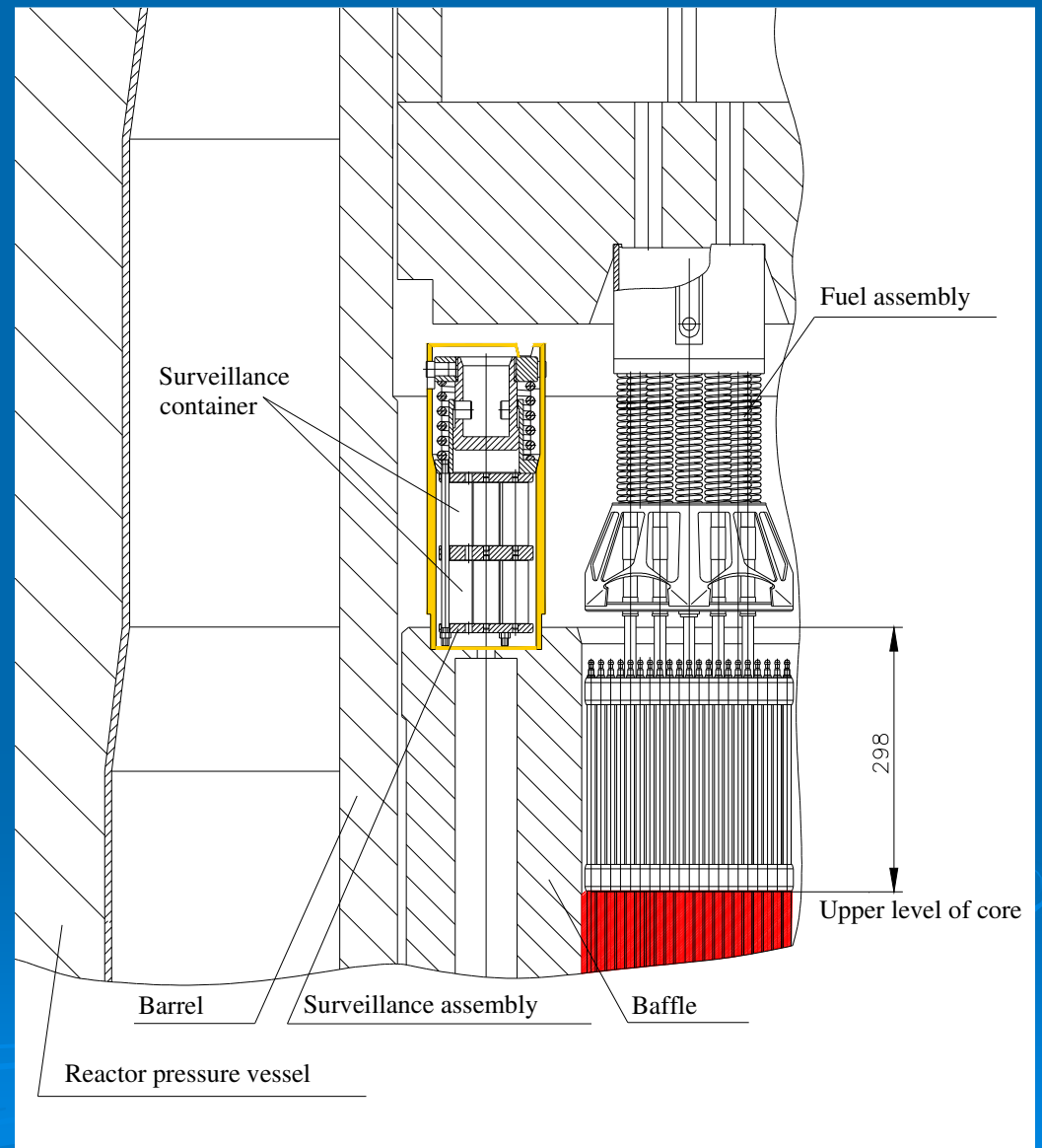


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LOCATION OF SURVEILLANCE ASSEMBLY IN VVER-1000

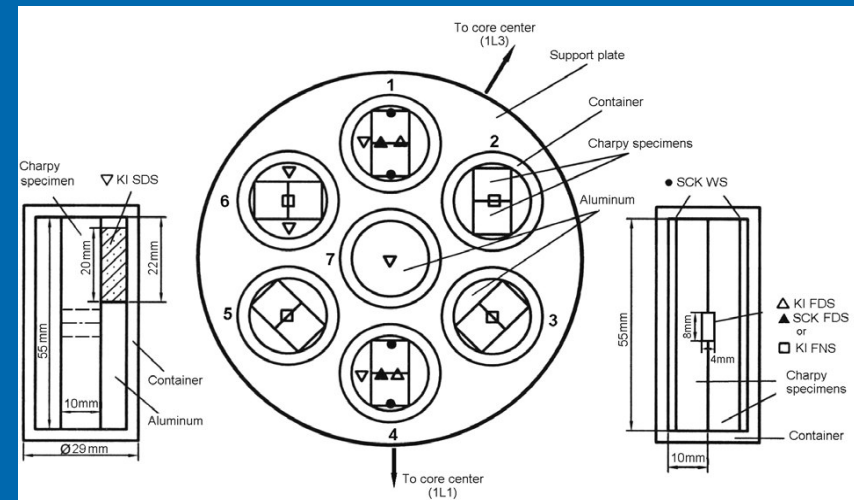
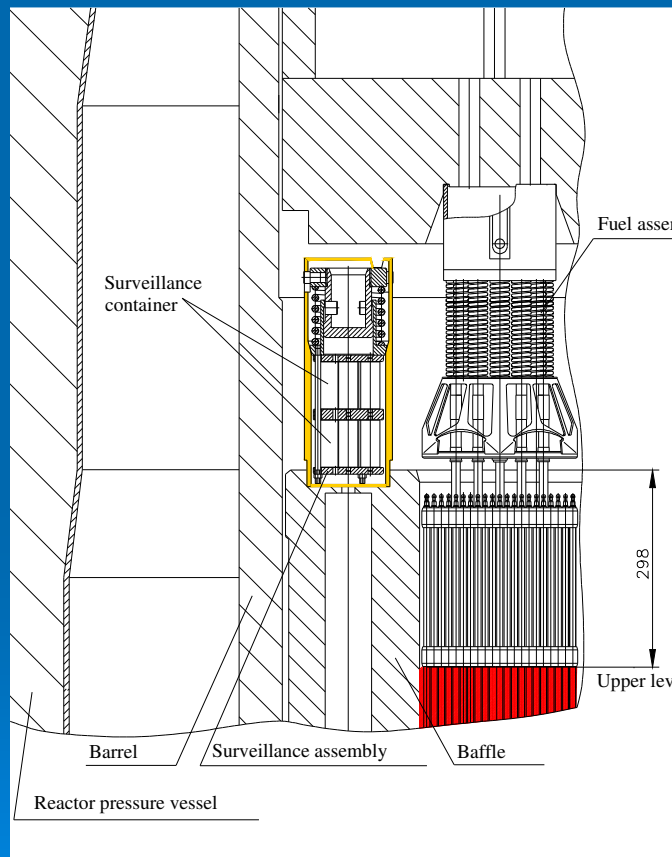


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EXPERIMENT ON BALAKOVO-1 VVER-1000 (DOSIMETRY OF SURVEILLANCE SPECIMENS)



SET		MONITOR					
		^{93}Nb	^{54}Fe	^{58}Ni	^{46}Ti	^{63}Cu	CoAl
●	SCK/CEN WS		✓				
▲	SCK/CEN FDS		✓	✓	✓	✓	
△	KI FDS	✓	✓	✓	✓	✓	✓
▼	KI SDS	✓	✓			✓	
□	KI FNS	✓	✓				

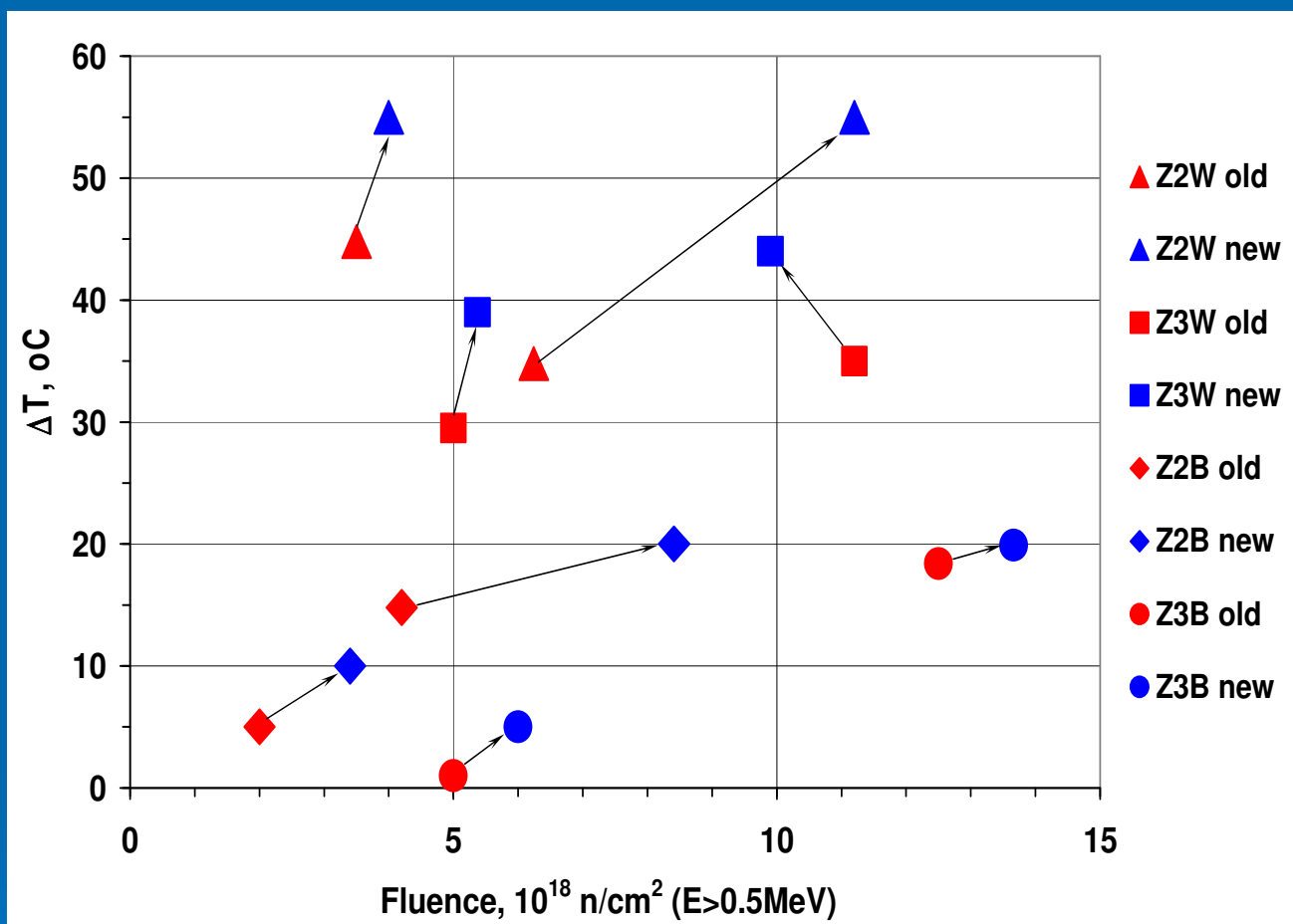


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SOME PRELIMINARY RESULTS OF VVER-1000 SURVEILLANCE SPECIMENS TESTING



*Z2 – Zaporozhie-2, Z3 – Zaporozhie-3,
B– base metal, W – weld,
old – before fluence reevaluation, new – after fluence reevaluation*

CONCLUSIONS

